

390



## ZINC INJECTION IN LIGHT WATER REACTORS TO MITIGATE STRESS CORROSION **CRACKING OF ALLOY 182 AND COLD-WORKED 316L STAINLESS STEEL**

Adrianna Mackiewicz<sup>1,2,\*</sup>, Stefan Ritter<sup>1</sup>, Kai Chen<sup>1</sup>, Hans-Peter Seifert<sup>1</sup>, Sannakaisa Virtanen<sup>2</sup>

1. Laboratory for Nuclear Materials, Nuclear Energy and Safety Division, Paul Scherrer Institute (PSI), Villigen, Switzerland 2. Department of Materials Science and Engineering, University of Erlangen-Nuremberg, Erlangen, Germany \*adrianna.mackiewicz@psi.ch



tests.



## **Oxide Film Formation under BWR Conditions**

590



If the oxide film is formed without Zn, the SCC mitigation effect seems to disappear.

The oxide film on Zn-treated specimens appears to be more uniform with smaller oxide crystals.

Hypothesis: Cr-rich inner oxide layer limit diffusion of Ni and Fe ions, that contributes to the oxide growth. They are known to occur through a solid state process. Zn-treated surface seems to have less vacancies in the inner Crrich layer.

Preferential oxidation of the GBs is visible on the OPS polished surfaces.

## OUTLOOK

Further investigations are planned: SCC growth tests, analytical electron microscopy (TEM, HR-STEM), X-ray photoelectron spectroscopy (XPS), electrochemical characterization, etc.

Acknowledgments: Presentation of a poster was made possible by the ZINC project carried out at PSI and financially supported by the Swiss Nuclear Safety Inspectorate (ENSI) and This project has received funding from the European Union's Horizon 2020 research and innovation program under the Marie Skłodowska-Curie grant agreement No 884104 (PSI-FELLOW-III-3i).

